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JMCT Monte Carlo Simulation Analysis of BEAVRS and SG-III Shielding

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Abstract. JMCT is a general purpose Mont Carlo neutron-photon-electron or coupled neutron/photon/electron transport code with a continuous energy and multigroup. The code has almost all functions of a general Monte Carlo code which include the various variance reduction techniques, the multi-level parallel computation of MPI and OpenMP, the domain decomposition and on-fly Doppler broadening, etc. Especially, JMCT supports the depletion calculation with TTA and CRAM methods. The input uses the CAD modelling and the calculated results use the visual output. The geometry zones, materials, tallies, depletion zones, memories and the period of random number are enough big for suit of various problems. This paper describes the application of the JMCT Monte Carlo code to the simulation of BEAVRS and SG-III shielding model. For BEAVRS model, the JMCT results of HZP status are almost the same with MC21, OpenMC and experiment. Also, we performed the coupled calculation of neutron transport and depletion in full power. The results of ten depletion steps are obtained, where the depletion regions exceed 1.5 million and 120 thousand processors to be used. Due to no coupled with thermal hydraulics, the result is only for reference. Finally, we performed the detail modelling for Chinese SG-III laser facility, where the anomalistic geometry bodies exceed 10 thousands. The flux distribution of the radiation shielding is obtain based on the mesh tally in case of Deuterium-Tritium fusion reaction. The high fidelity of JMCT has been shown.

1 INTRODUCTION

The high fidelity particle transport system JPTS(J Particle Transport System) has been developed for simulation of reactor full-core and shielding. This package is developed based on the three support framework JASMIN[1], JAUMIN and JCOGIN[2], where JASMIN is an adaptive structured mesh infrastructure, JAUMIN is an adaptive unstructured mesh infrastructure and JCOGIN is a parallel combinatorial geometry infrastructure. The JPTS package can do large scale parallel computation. Here, we mainly introduce a general purpose 3-D Monte Carlo transport code JMCT(J Monte Carlo Transport)[3]. Two models are chosen as test examples, where model one is the BEAVRS which is permitted by MIT Computational Reactor Physics Group in M&C2013 conference[4]. Another model is from the Chinese SG-III laser facility.

For BEAVRS model, we finish the detail modelling and simulation for the full core in hot zero power(HZP) status. 95% pins have less 1% standard deviation. The detailed pin-power density distribution, standard deviation etc. are shown. Then, we make the coupled calculation of neutron transport and depletion in full power status. Due to the memory resume too large, the simulation is done in case of 30/398 axial plane, where the depletion regions exceed 1.5 million. The simulation uses 120 thousand processors in Chinese TianHe-II supercomputer. Since the BEAVRS model involves the coupled with thermal hydraulics. At present, JMCT isn't coupled with thermal dynamic (being done), so the result is only for reference.

For SG-III model, the geometry is very complicated and irregular; the total geometry bodies exceed ten thousand. We order built some special body, such as optical instrument. The neutron and photon flux distributions of all building are given in case of the deuterium-tritium fusion reaction. The result has been used for the theory evidence of shielding design.

2 INTRODUCE OF JPTS PACKAGE AND JMCT MONTE CARLO CODE

2. 1 JPTS Package

JPTS package is developed by IAPCM. It contains the four applied codes JNuDa, JSNT, JMCT, JBURN and a suit of data libratory NuDa. Furthermore, the CAD pre-processor JLAMT and view post-processor TeraVAP are equipped(see Figure1). Where:

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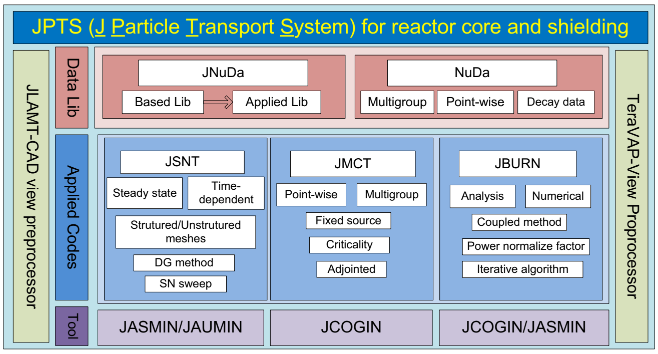

Fig. 1. JPTS package flow

Currently, JSNT S_N Code Support [5,6]
1) **Particle type**: neutron, photon or coupled of neutron and photon.
2) **Mode**: forward/adjoint.
3) **Problem**: fixed-source/criticality.
4) **Space**: 3D/2D, Cartesian /cylindrical geometry.
5) **Mesh**: non-uniform structured (JSNT-S)/unstructured mesh (JSNT-U).
6) **Energy**: multigroup, anisotropic P_N scattering (N=1, 3, 5).
7) **Parallelization**: massive parallel computing (space-angle parallelization).
8) **Algorithm**: acceleration algorithms (rebalance methods, multigrid methods).
9) **Input**: visualization modelling and automatic mesh generation, multiple choice of spatial discretion (TWD, DFEM ...).
10) **Output**: visualization analysis.
11) **Support Framework**: JASMIN[1].

Currently, JBURN Burnup Code Support
1) **Analysis**: Transmutation Trajectory Analysis (TTA).
2) **Numerical**: Chebyshev Rational Approximation Method (CRAM).
3) **Mode:** inner coupled with JMCT.
4) **Depletion regions:** >1 millions.
5) **Support Framework**: JCOGIN[2].

Currently, JNuDa Cross-Section Code Support
Continuous point-wise/Multigroup/Decay Data, where
1) Point-wise cross-section about 450 nucleus.
2) Multigroup library of 47/172 group for neutron and 20 groups for gamma.
3) Decay data (>1500 elements).

Currently, JLAMT Pre-processor Support[7]
1) **Geometry:** sphere, cylinder, rectangle, et al., some special geometrical body can be ordered.
2) **Repeat structure:** especial supporting the same geometry with the different material.

Currently, TeraVAP Post-processor Support
Scale: TB scale data and parallel visualization output.

2.2 JMCT Monte Carlo Code[3]

JMCT is a general purpose 3-D Monte Carlo transport code of neutron, photon, electron or coupled neutron/photon/ electron with the combinatorial geometry. Currently, JMCT support:
1) **Particle type**: neutron, photon, electron and coupled neutron/photon/electron.
2) **Mode**: forward/adjoint/burnup.
3) **Problem**: fixed-source/criticality.
4) **Space**: 3D combinatorial geometry.
5) **Energy**: continuous/multigroup (P_5).
6) **Source**: standard source/pin-by-pin source/user defined source.
7) **Tally**: point/surface/cell/mesh.
8) **Algorithms**: domain decomposition[8]/uniform tally density[9] /mesh tally and mesh windows. etc.
9) **Parallelization**: MPI (particle)+OpenMP **(**domain**)**.
10) **Input/Output**: CAD modelling and visualization.
11) **Temperture**: on-fly Doppler broadening[10].
12) **Fast critical search of boron concentration:** only one step.
13) **Tally types**: keff, point/surface/volume flux, energy deposition, power and reactivity etc.
14) **Support Framework**: JCOGIN[2].

3 TESTS

3.1 BEAVRS Model

3.1.1 Introduction of Model

The BEAVRS model was released by MIT Computational Reactor Physics Group in July 7, 2013 (www.crpg.mit.edu). It includes detailed specification of operating 4-loop Westinghouse PWR (3411MW), two cycles of measured data, HZP/full power data, fuel loads by assembly as built, three enriched fuels(1.6%, 2.4% and 3.1%). The detailed data is in reference[4]. Two cycles of measured data can be used to validate high-fidelity core analysis codes. The basic data is as following:
Fuel assemblies: 193.
Axial planes: 398.
Pins/assembly: 289(17×17, where 264 fuel pins and 25 guide tubes).
Total tally regions: 22,199,246 (193×17×17×1×398).
Total regions: 44,398,492 (193×17×17×2×398).
Requirement: ≤1% standard deviation for 95% fuel pin-powers.
The part results of MC21[11] and OpenMC[12] were presented in PHYSOR2014[13].

3.1.2 Simulated Results

(1) Results in HZP status

HZP status is simulated in 398 layers in axial direction. Due to the memory exceeding the limit of single core, the space domain is decomposed into 8 parts(figure 2(f)). Eight pin types(figure 2(a)) and nine types of assembles(figure 2(b)). The tally was for all pin fuel regions, the simulation tracks 4

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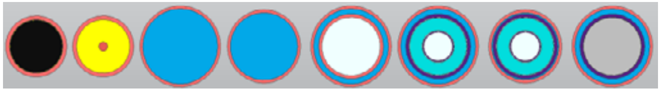
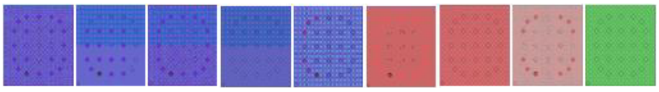
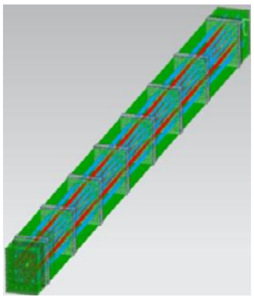
million neutrons each cycle, it discards 400 cycles of 1000 cycles. Table 1 shows the standard deviation distribution in 95% confidence level for all of pins. Table 2 shows the keff comparison of JMCT, OpenMC and MC21 in different location of control rods and boron concentrations. Table 3 shows the reactivity worth of control rods in 556K. Figure 3 shows the comparison of pin-power distribution, difference at axial elevation of peak power and the comparisons of the MC21 and JMCT powers in axial. The maximal difference is 3.17%. Figure 4 shows the detectors tallies in meter pipes between JMCT and experiment. The maximal difference is -14.77% in B13 assemble and minimum power assemble is -5.648% in L15 assemble. Figure 5 shows the axial power shape of the B13(maximal difference) and L15(minimum power assembles) between MC21 and JMCT as compared to experiment.



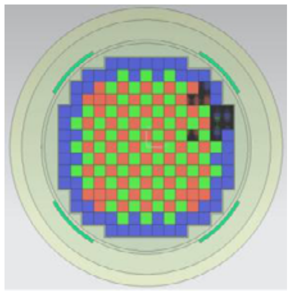
(a) Pins (eight types)



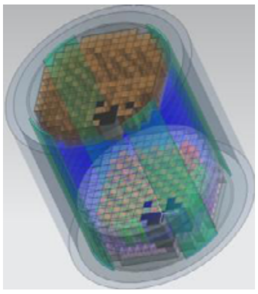
(b)assembles (nine types)



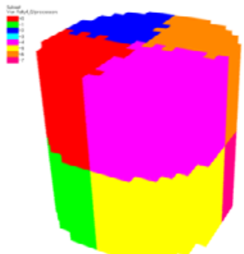
(c)grid



(d)core in radial



(e)full core



(f)2×2×2 domain decomposition

Fig.2. BEAVRS modelling by JLAMT

Table 1. Max and min pin error of flux and energy deposition

Count	MAX	MIN	95%	99%
Flux	0.0091	0.00118	<0.00332	<0.00423
Energy deposition	0.01933	0.00254	<0.0075	<0.00955

Table 2. keff comparison in different control rod statuses and boron concentration

HZP Critical Boron Evaluation	Boron Concentration	JMCT (95% confidence leave)	OpenMC (95% confidence leave)	MC21 (95% confidence leave)
ARO	975	1.000479±0.000030	0.99920±0.00004	0.9992614±0.000004
D in	902	1.002174±0.000030	1.00080±0.00004	
C,D in	810	1.001419±0.000032	1.00023±0.00005	
A,B,C,D in	686	0.9999172±0.000032	0.99884±0.00004	
A,B,C,D,SE,SD,SC in	508	0.9983806±0.000032	0.99725±0.00004	

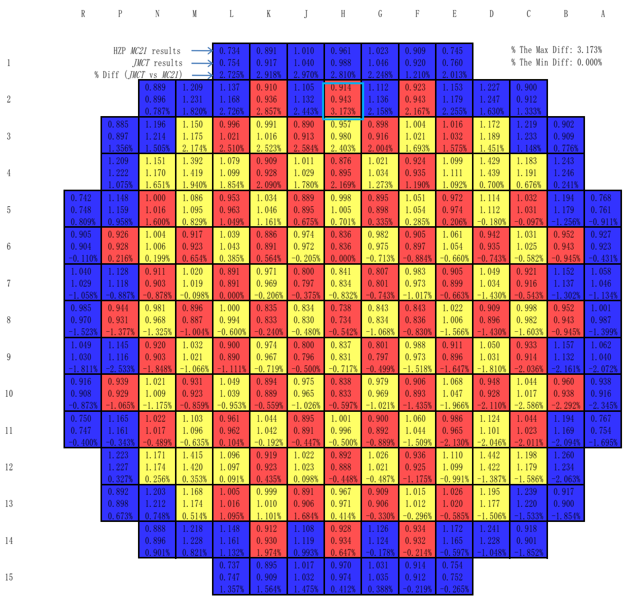


Fig. 3. Comparison of pin power distribution and difference at axial elevation of peak power between MC21 and JMCT

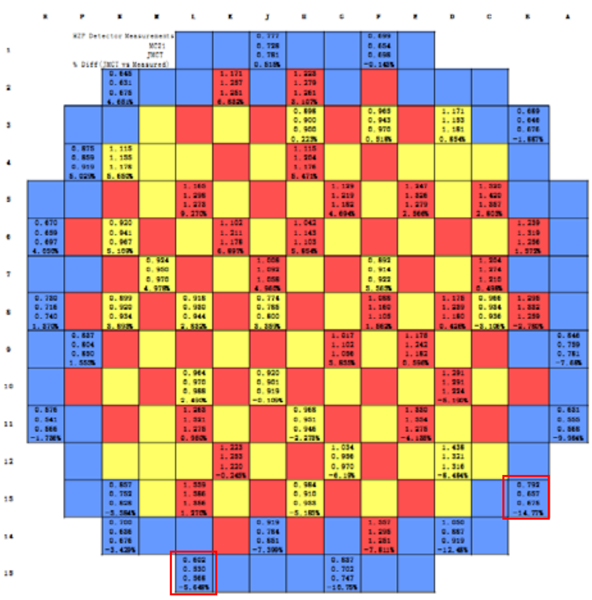


Fig. 4. Comparison of the detectors tallies in meter pipes

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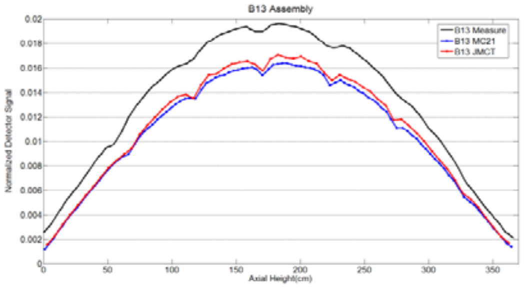
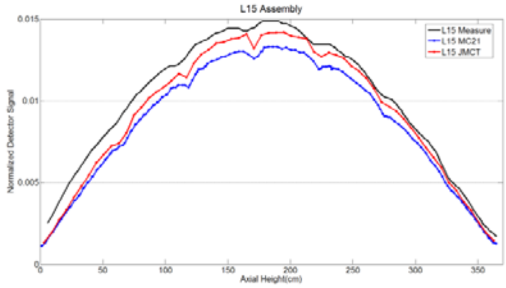
Table 3. Comparison of reactivity worth of control rod in different statuses and boron concentration

HZP Bank worth	Boron	Measure	MC21	OpenMC	JMCT
D	938.5	788	773	771±6	770±6
C with D in	856	1203	1260	1234±7	1258±6
B with D,C in	748	1171	1172	1197±7	1162±6
A with D,C,B in	748	548	574	556±6	578±6
SE with D,C,B,A in	597	461	544	501±6	543±6
SD with D,C,B,A,SE in	597	772	786	844±6	781±6
SC with D,C,B,A,SE,SD in	597	1099	1122	1049±6	1107±6



(a)axial power shape in B13 assembly
(with maximal difference)

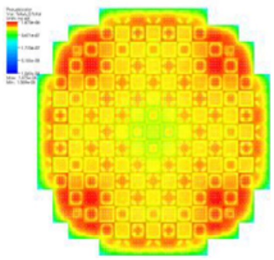


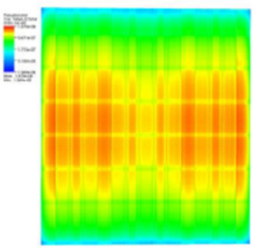
(b)axial power shape in L15 assembly
(with minimum power)

Fig. 5. The axial power shape comparison of MC21, JMCT and experiment

(2)Result in Full Power Status

The coupled neutron transport and depletion is run in 30/398 axial planes, where the depletion region is up to 1528560 (193×264×30). The space domain is also decomposed into 8 parts. It takes about one hour with 120,000 cores on Chinese TianHe-II computer. The result of the tenth steps is obtain. Figure 6 shows the power distribution of some status. Due to the JMCT no coupled with thermal hydraulics, the result is only for reference. For full simulation of BEAVRS model, due to the large memory resume, we predict that the goal need a long time and impossible before 2018.


(a)pin power in radial


(b)pin power in axial

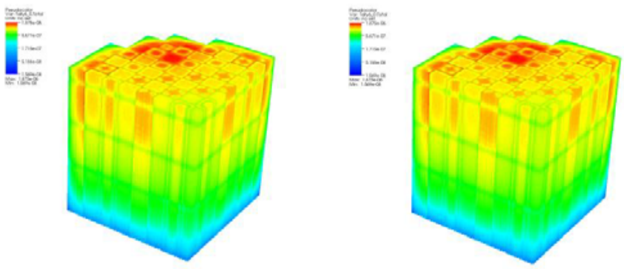
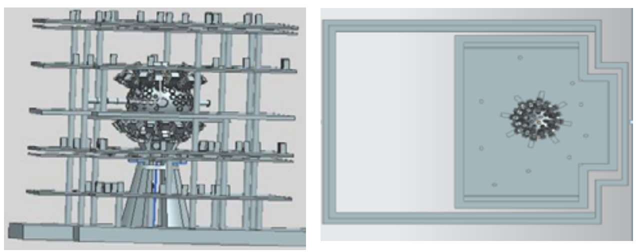

(c)pin power distribution in other status

Fig.6. Pin power distributions in full power

3.2 Chinese SG-III Laser Model

Chinese SG-III laser device is with 48 laser beams and size in 45m×45m×53m. Diameter of target chamber is 6 m. Power is 50 TW. It is applied to drive the nuclear fusion reaction by the laser energy. Figure 7 shows the modelling by JMCT pro-processor JLAMT, where figure 7(a)(b) show the locations of tally floors and figure 7(c) shows the building (six floors in ground). The tally is for all floors (seven floors in total). Mesh tally does and it has about 0.63 million meshes. The 0.4 billion neutron histories are simulated by 1024 cores. The 3.1 CPU hours are taken. Where the source is a 14.1 MeV deuterium and tritium(D-T) neutron point source. Figure 8(a)-(b) gives the neutron and photon flux distributions in the base of the fourth floor. Figure 8(c) gives the energy distributions of each floor. Figure 8(d) gives a part of flux distribution in building.


(a)location of the tally floors

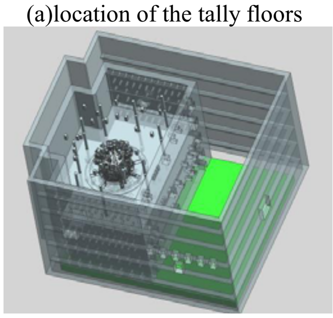

(b)building of SG-III device

Fig.7. Section of SG-III laser devic

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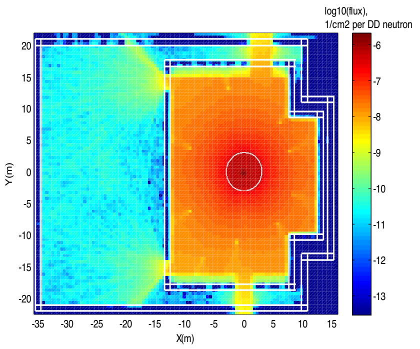
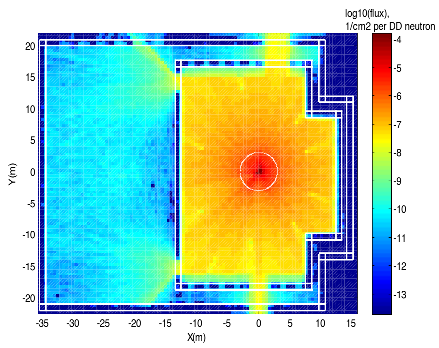
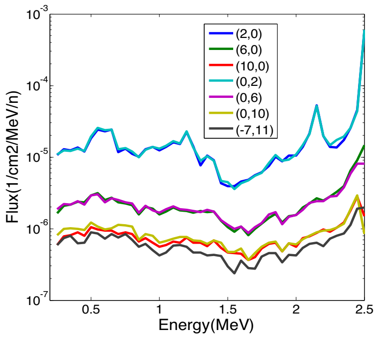


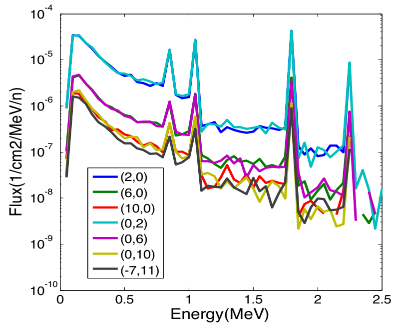
(a)neutron flux in horizontal plane



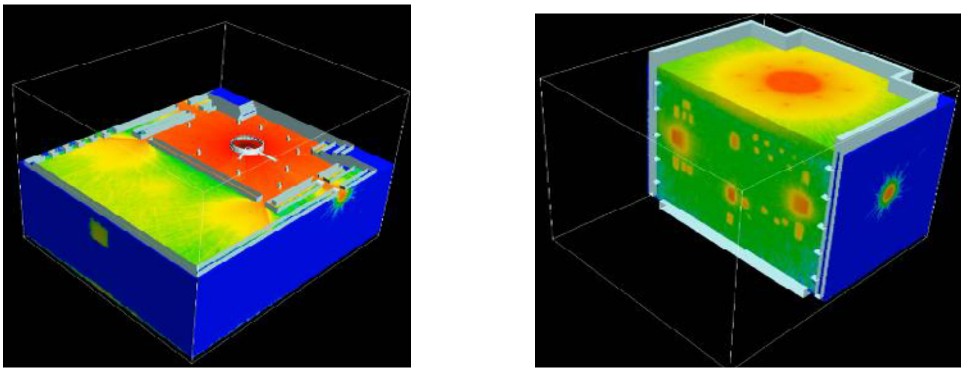
(b)photon flux in horizontal plane



(c)neutron energy spectrum in each floor



(d)photon energy spectrum in each floor



(e)flux distribution of full building (moving)

Fig.8. Flux and energy spectrums in D-T reaction for SG-III model

4 Conclusion

JMCT Monte Carlo code is developed and with the capability of the full-core pin-by-pin simulation. It well suits to simulate the large nuclear power reactors and radiation shielding of some large facilities. At present, JMCT still exist no symmetry in power distribution after several burnup steps. We are analysing the reason. On the other hand, the depletion complicating uncertainty quantification and propagation of error will be considered in our next work. Furthermore, it needs to find some new methods to reduce the computational fee. At present, some challenges still exist in simulation of the BEAVRS model. Some new algorithms are being developed.

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